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1. Introduction

The activation products in a reactor’s primary coolant loop is a main reason why Nuclear Power Plants (NPPs) use a chain of two or three coolant loops linked by heat exchangers, but isolated against matter exchange. However, during routine operations, damages are likely to occur, and the formation of micro-cracks represents a way of leakage for activation products. Through the same way, also leakage of material directly from the reactor core can take place: fission products and several isotopes of actinides, $^{235}$U, $^{238}$U, $^{239}$Pu and those heavier isotopes produced by single and multiple neutron capture on them. The explosion of a nuclear bomb leads itself to releases of radioactive material, which, in this case, could be widespread in the environment up to a global scale.

Moreover, undeclared nuclear activities and/or illegal use and transport of nuclear fuel can be detected through determination of the isotopic ratios of U and Pu in such samples. As all the activities related to nuclear technology have been, since the second half of the past century, responsible of changes in the concentration of the anthropogenic radionuclides over global scale, it would be not inappropriate referring to a further Global Environmental Change which is recognizable within every environmental matrix: soil, superficial and ground-waters and living organisms. The outcome of the interactions of ionizing particles with living matter is, already, unluckily, well known, that’s why a number of studies have been flourishing in order to assess the radiological risk, by analyzing the environmental pathways of radionuclides and their behavior within the human food chain. In parallel, accurate assessment and monitoring of every source of radioactive contamination are required from the point of view of the prevention from this new scenario of radiological exposition (Quinto, 2007). Quantifying the releases and tracing their dispersion in the environment has traditionally been the task of Alpha Spectrometry (AS (O’Donnell et al., 1997; Sanchez et al., 1992) or, more recently, of Inductively Coupled Plasma Mass Spectrometry (ICP-MS) (Ketterer et al., 2003; Ketterer & Szechenyi, 2008; Wyse et al., 2001) or Thermal Ionization (TI-MS) (Beasley et al., 1998; Richter et al., 1999). Although these are mature methodologies, each has its limitations, which are largely surmounted by the relatively new technique of Accelerator Mass Spectrometry (AMS) (Fifield, 2008; Steier et al., 2010). In the chapter the origin of actinides is discussed as well as the potential of actinides to serve as a tracer for geomorphologic processes and as sensitive...
fingerprint of releases from nuclear facilities (Quinto et al., 2009; Sakaguchi et al., 2009; Steier et al., 2008).

Moreover the sensitivity of the different actinides measurements method and the peculiarity of the AMS technique with respect to AS and CMS techniques are illustrated. Furthermore the principles and methodology of heavy-element AMS as applied to U and Pu isotopes are described, and the ways in which these have been implemented in various laboratories around the world are discussed with particular attention to the CIRCE (Center for Isotopic Research on Cultural and Environmental Heritage) and ANU (Australian National University) accelerator facilities (Fifield et al., 1996; Terrasi et al., 2007).

2. Origin and detection of actinides

2.1 Sources of contamination

The discovery of the fission properties of $^{235}$U and $^{239}$Pu and the capability to sustain chain fission reactions, have opened the way to several applications of nuclear energy. Nuclear Power Plants are used to generate thermal, and then electric, power, while Nuclear Weapons use the destructive effects of supercritical chain reactions. Both of these applications of nuclear energy, together with all the activities correlated with production, transport and reprocessing of nuclear fuel, lead to releases of a wide range of radioactive elements in the environment.

Free neutrons produced by fission are partly adsorbed by other nuclei of fissile material, allowing, in the appropriate conditions, the self-sustainability of the chain reaction, and partly adsorbed by other elements in the air and the ground, during fission in nuclear explosions, and by structural and cladding materials and coolant circuits, during the exercise phase of NPP. The production of activation products in a reactor’s primary coolant loop is a main reason why NPPs use a chain of two or three coolant loops linked by heat exchangers, but isolated against matter exchange. However, during routine operations, damages are likely to occur, and the formation of micro-cracks represents a way of leakage for activation products. Through the same way, also leakage of material directly from the reactor core can take place: fission products and several isotopes of actinides, $^{235}$U, $^{238}$U, $^{239}$Pu and those heavier isotopes produced by single and multiple neutron capture on them. The input of these radionuclides in the outer coolant circuit results in their release in the environment within airborne and liquid effluents during the routine running of a NPPs. Of course, other ways of leakage would be accidents leading to uncontrolled explosions of the reactor or any kind of exceptional damage at various structural devices. The diffusion area of contamination from routine exercise of NPPs is restricted to a local scale, affecting the surroundings, the river or artificial channel, which provides and drains the reactor cooling water and the following areas connected with the river water circulation (Quinto, 2007). On the some grounds, the above radionuclides can be dispersed in the environment by the explosions of nuclear weapons in the atmosphere or underground.

Schematically it is possible to divide the sources of contamination into large and small scale contamination:

1. large scale source of radionuclide contamination includes the explosion of a Nuclear bomb. The contamination could be widespread in the environment up to a global scale. Several atmospheric nuclear tests conducted between 1945 and 1963 by the US and Soviet Union, by UK between 1952 and 1963, by France until 1974 and by the People’s Republic of China until 1980, projected their debris into the stratosphere, from where, due to the global wind circulation, they have been deposited around the world within what is called global fall...
Table 1. Half-lives, $T_{1/2}$ and decay mode of Uranium and Plutonium isotopes in year, y, units.

<table>
<thead>
<tr>
<th>Actinides</th>
<th>$T_{1/2}(y)$</th>
<th>Emitted radiation</th>
</tr>
</thead>
<tbody>
<tr>
<td>$^{234}\text{U}$</td>
<td>$2.5 \cdot 10^3$</td>
<td>$\alpha$</td>
</tr>
<tr>
<td>$^{235}\text{U}$</td>
<td>$7.0 \cdot 10^9$</td>
<td>$\alpha$</td>
</tr>
<tr>
<td>$^{236}\text{U}$</td>
<td>$23.0 \cdot 10^6$</td>
<td>$\alpha$</td>
</tr>
<tr>
<td>$^{238}\text{U}$</td>
<td>$4.5 \cdot 10^9$</td>
<td>$\alpha$</td>
</tr>
<tr>
<td>$^{238}\text{Pu}$</td>
<td>$88.0$</td>
<td>$\alpha$</td>
</tr>
<tr>
<td>$^{239}\text{Pu}$</td>
<td>$2.4 \cdot 10^4$</td>
<td>$\alpha$</td>
</tr>
<tr>
<td>$^{240}\text{Pu}$</td>
<td>$6.6 \cdot 10^3$</td>
<td>$\alpha$</td>
</tr>
<tr>
<td>$^{241}\text{Pu}$</td>
<td>$14$</td>
<td>$\beta^-$</td>
</tr>
<tr>
<td>$^{242}\text{Pu}$</td>
<td>$3.8 \cdot 10^2$</td>
<td>$\alpha$</td>
</tr>
<tr>
<td>$^{243}\text{Pu}$</td>
<td>$5.7 \cdot 10^{-4}$</td>
<td>$\beta^-$</td>
</tr>
<tr>
<td>$^{244}\text{Pu}$</td>
<td>$82.0 \cdot 10^9$</td>
<td>$\alpha$</td>
</tr>
</tbody>
</table>

2.2 Different contamination sources

The relative concentrations of plutonium and uranium isotopes depend on the nature of the source material and on its subsequent irradiation history; all these sources of contamination do not give the same contributions of contamination. As it will be shown in the following, useful tools to solve among different contributions are the isotopic ratios: $^{236}\text{U}/^{238}\text{U}$, $^{240}\text{Pu}/^{239}\text{Pu}$, $^{241}\text{Pu}/^{239}\text{Pu}$, $^{242}\text{Pu}/^{239}\text{Pu}$ and $^{238}\text{Pu}/^{239}+^{240}\text{Pu}$.

Obviously, given that the relative concentrations of plutonium and uranium isotopes depend on the nature of the source material and on its subsequent irradiation history, all these sources of contamination do not give the same contributions of contamination. As it will be shown in the following, useful tools to solve among different contributions are the isotopic ratios: $^{236}\text{U}/^{238}\text{U}$, $^{240}\text{Pu}/^{239}\text{Pu}$, $^{241}\text{Pu}/^{239}\text{Pu}$, $^{242}\text{Pu}/^{239}\text{Pu}$ and $^{238}\text{Pu}/^{239}+^{240}\text{Pu}$.

Table 1 shows the half lives of the relevant isotopes of U and Pu.
\[ ^{238}\text{U} \xrightarrow{n} ^{239}\text{U} \xrightarrow{\beta^-} ^{239}\text{Np} \xrightarrow{\beta^-} ^{239}\text{Pu} \xrightarrow{n} ^{240}\text{Pu} \xrightarrow{n} ^{241}\text{Pu} \xrightarrow{n} ^{242}\text{Pu} \ldots \]

In weapon test fallout, the ratio \(^{240}\text{Pu}/^{239}\text{Pu}\) varies depending on the test parameters in the range of 0.10-0.35. The average for the Northern hemisphere is about 0.18, (Koide et al., 1985).

Significantly different values, in the range 0.035-0.05, are found in Mururoa and Fangataufa atoll sediment, because of the particular nature of French testing, (Chiappini et al., 1999) and (Hrneceka et al., 2005).

- In nuclear reactors, as mentioned before, due to the different composition of fuels, uranium enrichment and burn-up degree, characteristic relative abundances of plutonium isotopes will be obtained: \(^{240}\text{Pu}/^{239}\text{Pu}\) increases with irradiation time, which, in turn affects \(^{238}\text{Pu}/^{239}+^{240}\text{Pu}\).

\(^{239}\text{Pu}\) is produced by neutron capture from \(^{237}\text{Np}\), which is itself produced by two successive neutron captures from \(^{235}\text{U}\):

\[ ^{235}\text{U} \xrightarrow{n} ^{236}\text{U} \xrightarrow{n} ^{237}\text{U} \xrightarrow{\beta^-} ^{237}\text{Np} \xrightarrow{n} ^{238}\text{Np} \xrightarrow{\beta^-} ^{238}\text{Pu} \]

or via the fast-neutron induced \(^{238}\text{U}(n,2n)^{237}\text{U}\) reaction:

\[ ^{238}\text{U}(n,2n)\xrightarrow{} ^{237}\text{U} \xrightarrow{\beta^-} ^{237}\text{Np} \xrightarrow{n} ^{238}\text{Np} \xrightarrow{\beta^-} ^{238}\text{Pu} \]

The ratio \(^{238}\text{Pu}/^{239}+^{240}\text{Pu}\) is useful to resolve between different sources in case they show similar \(^{240}\text{Pu}/^{239}\text{Pu}\), e.g., irradiated nuclear fuel in a PWR (Pressurized Water Reactor) with 7-20% of \(^{235}\text{U}\) and burn-up 1.4-3.9 GW·d (GWatt-day) reaches \(^{240}\text{Pu}/^{239}\text{Pu}\) isotopic ratios of 0.13, a value, that could be ascribed also to global fall out. On the other side, these two sources show quite different \(^{238}\text{Pu}/^{239}+^{240}\text{Pu}\) activity ratio, 0.025-0.04 for the global fallout and 0.45 for that nuclear fuel, (Quinto, 2007).

- Another valuable tool to identify a nuclear reactor origin of a radionuclide contamination is \(^{236}\text{U}/^{238}\text{U}\) isotopic ratio. The dominant \(^{236}\text{U}\) mode of formation is the capture of a thermal neutron by \(^{235}\text{U}\), a secondary contribution being the alpha decay of \(^{240}\text{Pu}\). Its concentration in nature has been heavily increased as a consequence of irradiation of enriched uranium in nuclear reactors. Several orders of magnitude of difference between the \(^{236}\text{U}/^{238}\text{U}\) isotopic ratios in naturally-occurring uranium (10\(^{-9}\) to 10\(^{-13}\)) and in spent nuclear fuel (10\(^{-2}\) to 10\(^{-4}\)) imply that also a small contamination from irradiated nuclear fuel in natural samples is able to increase significantly the \(^{236}\text{U}/^{238}\text{U}\) ratio measured in the whole sample.

2.3 Needs for actinides monitoring

The nuclear safeguard system used to monitor compliance with the Nuclear Non-proliferation Treaty relies to a significant degree on the analysis of environmental samples. Undeclared nuclear activities and/or illegal use and transport of nuclear fuel can be detected through determination of the isotopic ratios of U and Pu in such samples. Accurate assessment and monitoring of every source of radioactive contamination are required from the point of view of the prevention from radiological exposure.

Both the operations of decommissioning of the existing NPPs and the possible future operation of new plants demand accurate investigations about the possible contamination by radioactive releases of nuclear sites and neighboring territory and of structural...
materials of the reactors. The monitoring activity of surveillance institutions uses assessed radiometric techniques, but more and more ultrasensitive methodologies for the detection and quantification of ultralow activity radionuclides is requested at international level. Most of U and Pu isotopes are long lived alpha emitters with very low specific activity: their detection and the measurement of their concentration and isotopic abundance demands very high sensitivity, so that they are included among the so called “hardly detectable” radionuclides. As it will be shown in the following, the required sensitivity is often not achieved using conventional analytical techniques, such as counting of the radiation emitted in the decay or conventional mass spectrometry. The main task of the present work is to illustrate an ultrasensitive methodology for the detection of ultralow level radionuclides belonging to the actinides subgroup of the periodic table. The method is based on a combination of AS and AMS: the reason for such a combination lies in the fact that it may be necessary to be able to measure the Pu isotopes at the fg level and the U isotopes where the total uranium content may be at the ng level or with a sensitivity as low as $10^{-13}$ in the measurement of the $^{236}$U/$^{238}$U isotopic ratio in samples incorporating a total of about 1 mg of U. AMS will be shown to be the only technique able to achieve such a sensitivity together with unparalleled suppression of molecular isobaric interferences for the detection of rare isotopes of elements with (quasi)stable isotopes many orders of magnitude more abundant, such as U. Nevertheless, the measurement of $^{238}$Pu abundance is heavily suffering interference from the atomic isobar $^{238}$U, about seven orders of magnitude more abundant, and cannot be achieved by any mass spectrometric technique; on the other hand ultra-low activity AS can isolate this isotope, while alpha particles from the decay of $^{239}$Pu and $^{240}$Pu cannot be energetically resolved. Combination of the two techniques provides the determination of the abundances of the full suite of Pu isotopes. Moreover, AS plays an important role also for the calibration of the spikes used as carriers for the AMS measurements and as overall cross check of the employed methodologies. An important role in pursuing the goal of ultrasensitive detection of actinide isotopes is played by the sample preparation procedures, which has to be performed in a very clean environment with ultralow contamination. The procedure to be setup will be able to isolate the elements of interest and produce samples in the form suitable for both AS and AMS. In the first case very thin and uniform layers have to be achieved, while purification respect to elements which can produce molecular interferences is of paramount importance for AMS. Preliminary sampling and conditioning of a properly representative sample; uranium and plutonium are separated from the sample following a systematic chemical protocol of pre-enrichment/separation; fractions of U and Pu are purified from every possible element that could cause radiochemical interference to AS; fractions of U and Pu must be converted into useful chemical and physical-chemical forms (De Cesare, 2009; Quinto et al., 2009; Wilcken et al., 2007). Finally, besides the application of the developed technique to the assessment of actinide contamination of the NPP site and plant, a more general objective is to provide an ultrasensitive diagnostic tool for a variety of applications to the national and international community. Applications range across a broad spectrum. Isotopes of plutonium are finding application in tracing the dispersal of releases from nuclear accidents and reprocessing operations, in studies of the biokinetics of the element in humans, and as a tracer of soil loss and sediment transport. $^{238}$U has also been used to track nuclear releases, but additionally has a role to play in nuclear safeguards and in determining the extent of environmental contamination in modern theaters of war due to the use of depleted uranium weaponry.
2.4 Alpha-spectroscopy and mass spectroscopy

$^{236}\text{U}$ and $^{239}\text{Pu}$ are present in environmental samples at ultra trace levels ($^{236}\text{U}$ concentration is quoted to be in the order of pg/kg or fg/kg and $^{239}\text{Pu}$ around 100 pg/kg) and are long-lived radionuclides (Perelygin & Chuburkov, 1997). If one considers alpha-spectroscopy for the detection of $^{239}\text{Pu}$, assuming an efficiency of 50\% and a counting time of one month, one gets 64 counts (with a statistical uncertainty of 12\%) with a total activity of 50 $\mu$Bq, which correspond to about 40 million atoms, or about 15 fg. In addition, alpha-particle counting is unable to resolve the two most important plutonium isotopes, $^{239}\text{Pu}$ and $^{240}\text{Pu}$, because their alpha-particle energies differ by only 11 keV in 5.25 MeV. Hence, the information on their isotopic ratio readily difficult to extract. The $2.10^6$ y half-life of $^{236}\text{U}$ limits the utility of alpha-particle spectroscopy for this isotope.

For the detection of such small amounts one can exploit the sensitivity of mass spectrometric techniques. Conventional Mass Spectrometry, CMS, methods give information on the $^{240}\text{Pu}/^{239}\text{Pu}$ ratio, and potentially have higher sensitivity than alpha-particle counting with values as low as 1 fg, but are sensitive to molecular interferences. Both $^{236}\text{U}$ and $^3\text{Pu}$ isotopes have been measured using either Thermal Ionization (TI-MS) or Inductively Coupled Plasma (ICP-MS) positive ion sources. For plutonium isotopes, abundance sensitivity is not a problem due to the absence of a relatively intense beam of similar mass. Molecular interferences such as $^{238}\text{U}^-$, $^{208}\text{Pb}^{31}\text{P}$, etc. may be a problem (Fifield, 2008). For uranium, isotope variability both in the molecular ($^{238}\text{U}^-$) and in tail contributions of main beam of $^{238}\text{U}$ limits the sensitivity of ICPMS to $^{236}\text{U}/^{238}\text{U}$ ratios of $\sim 10^{-7}$. TIMS ion sources, on the other hand, produce both much lower molecular beams and much less beam tail and so a sensitivity of $\sim 10^{-10}$ in the $^{236}\text{U}/^{238}\text{U}$ ratio is reached.

So that, the measurements of these isotopic ratios requires the resolution of mass spectrometric techniques, but only AMS allows the sensitivity needed e.g. $^{230}\text{U}/^{238}\text{U}$ ratios of $\sim 10^{-13}$, 0.1 fg of $^{236}\text{U}$ with about 1 mg of U, as well as for the $^{239}\text{Pu}$. Although AMS has advantages over the other techniques for $^{239,240,242,244}\text{Pu}$, there are two other isotopes, $^{238}\text{Pu}$ and $^{241}\text{Pu}$, which are of interest in some applications. Since the concentration of $^{238}\text{U}$ is seven orders of magnitude higher than that of $^{238}\text{Pu}$, no chemical procedure is efficient to separate uranium and plutonium fractions to allow the mass spectrometric measurement of $^{238}\text{Pu}$. So alpha-spectroscopy remains the only suitable technique for the measurement of $^{238}\text{Pu}$ concentration. The $\beta^-$ emitter $^{241}\text{Pu}$ can be measured with either AMS or with liquid scintillation counting. Its short half-life of 14 years results, however, in higher sensitivity for the latter (Fifield, 2008).

2.5 AMS of actinides isotopes

Actinides AMS measurements were pioneered at the IsoTrace laboratory in Toronto (CA) (Zhao et al., 1994; 1997), where the $^{236}\text{U}$ content in an U ore was determined using the 1.6 MV AMS system. Moreover, the relative abundances of Pu isotopes were measured at 1.25 MV. Then, at the Australian National University (AUS) (Fifield et al., 1996; 1997) the utilization of a higher terminal voltage (4 MV) allowed to improve the sensitivity of the method, both for the detection limit as the minimum detectable number of U atoms in the sample, and for the lower limit of isotopic ratio measurable in samples at high concentration. Similar detection system have been developed at the Vienna Environmental Research Accelerator (VERA - AU) (Steier et al., 2002), at the Lawrence Livermore National Laboratory (LLNL - USA) (Brown et al., 2004), at the Australian Nuclear Science and Technology Organisation (ANSTO - AUS) (Hotchkis, 2000), at much lower energies at the Eidgenössische Technische
Hochschule - ETH in Zurich (CH) (Wacker et al., 2005), at Munich facility (GE) (Wallner et al., 2000) and at the accelerator of Weizmann Institute, Israel (Berkovits et al., 2000). New AMS actinides line based on 1MV and 3 MV tandems have recently been and will be installed, respectively, in Seville (Spain) and in the Salento (Italy). In both cases, they will be upgraded to perform actinides AMS measurements, being the injection and the analyzing magnets overdimensioned.

Two recent review papers ( Fifield, 2008; Steier et al., 2010) summarize the results obtained in the laboratories active in the fields of actinides AMS. Summarizing, the two systems aiming to the best isotopic ratio sensitivity (ANU and VERA) have shown that it is possible to reach a sensitivity of $10^{-13}$ for $^{236}$U in samples including about 1 mg of U. The ANSTO and LLNL laboratories quote a sensitivity respectively of about $10^{-8}$ and $10^{-9}$ with U amounts of the order of 1 ng. In the case of plutonium, there is no stable abundance isotopes available; the plutonium isotopic ratio is not a problem and a $^{239}$Pu concentration background of about 0.1 fg ($2.5 \times 10^5$ atoms) is achieved, limited by the process blank count rate. In both cases these limits surmount by several orders of magnitude alpha spectrometry and conventional mass spectrometry. In nature, U stable abundant isotopes exist. For that reason, the sensitivity limit for the isotopic ratio depends on the U concentration in the sample. Thus, the AMS task is, for environmental samples, to push the sensitivity in the isotopic ratio measurement down to natural abundances ($^{236}$U/$^{238}$U$\approx$10$^{-9}$-10$^{-13}$) in samples with sizeable amounts of U ($\sim$ 1 mg). On the other hand, for anthropogenically influenced samples, the required sensitivity for the measurement of the isotopic composition is alleviated, but significantly smaller amounts of U have to be used (down to 1 ng). For Pu, where no stable isotope interferences are present, the goal is the maximum possible detection efficiency, allowing few hundred counts from less than 1 million atoms in the sample.

The CIRCE laboratory is one of the few systems in the world able to perform such a measurement (De Cesare et al., 2010a) and the only one in Italy. Moreover it is 1 order of magnitude higher (De Cesare et al., 2010b) with respect to the 2 systems (ANU and VERA) providing the best $^{236}$U/$^{238}$U isotopic ratio sensitivity of $10^{-13}$, in samples including about 1 mg of U; it has low uranium contamination background, less then 0.4 µg of $^{236}$U (De Cesare et al., 2011). The CIRCE actinides group aims to reach and to exceed the isotopic ratio sensitivity goal with the upgrade: the utilization of a TOF system and, in case, the installation of a magnetic quadrupole doublet. Regarding the Plutonium background results, the CIRCE is one of the best systems in the world (De Cesare, 2009).

3. AMS facilities

In this paragraph the facilities where the author was mainly involved will be illustrated: CIRCE and ANU AMS systems.

3.1 CIRCE system

CIRCE is a dedicated AMS facility based on a 3MV-tandem accelerator (Terrasi et al., 2007). In contrast to many nuclear physics applications, the pre-treated sample material (a few mg is pressed into a 1 mm diameter Al cathodes and put in the ion source) itself is analyzed by two mass spectrometers which are coupled to the tandem accelerator. A schematic layout of the CIRCE facility is shown in Fig. 1.

The caesium sputter ion source is a 40-sample MC-SNICS (Multi Cathode Source for Negative Ions by Cesium Sputtering). A total injection energy of 50 keV is used; 50-300 nA $^{236}$U$^{16}$O$^{-}$ molecules are energy selected by a spherical electrostatic analyzer (nominal bending radius
CIRCE Accelerator

Fig. 1. Schematic layout of the CIRCE accelerator and of CIRCE accelerator upgrade with the actinides line and also the ERNA separator line, Astro line, besides the $^{14}$C original line: the switching magnet already inserted and the start and the stop TOF-E detector not yet inserted. FC denotes Faraday Cup (LFC in the actinides line is Last Faraday Cup), C denotes the Collimator in the heavy isotope line and the arrows indicate a slit system. ERNA is the acronym of European Recoil separator for Nuclear Astrophysics.

$r = 30 \text{ cm, plate gap}= 5 \text{ cm}$ which cuts the sputter low energy tail of the beam, with a bending angle of $\pm 45^{\circ}$ and it is operated up to $\pm 15 \text{ kV}$. The maximum electric field strength is $6 \text{ kV/cm}$, resulting in an energy/charge state ratio of $90 \text{ keV/q}$. The $90^{\circ}$ double focusing Low Energy (LE) injection magnet ($r = 0.457 \text{ m, vacuum gap}= 48 \text{ mm, } ME/q^2 = 15 \text{ MeV amu/e}^2$) allows high resolution mass analysis for all stable isotopes in the periodic table, mass resolution is $M/\Delta M \sim 500$ with the object and image slits set to $\pm 1 \text{ mm}$, (De Cesare et al., 2010a). The insulated stainless steel chamber (MBS) can be biased from $0 \text{ kV}$ to $+15 \text{ kV}$ for beam sequencing (e.g. between $^{238}\text{U}^{5+}$ at $2.875 \text{ MV}$).

The accelerator is contained inside a vessel filled with sulphur hexafluoride ($\text{SF}_6$) at a pressure of about 6 bar. Two charging chains supply a total charging current to the terminal; about 100 $\mu \text{A}$ are delivered to the terminal for operation at 3.000 MV. Stabilization is achieved by GVM feedback on the charging system high voltage supply; the long term stability is about $1 \text{ kV peak-to-peak}$. At the terminal the ions lose electrons in the gas stripper, where Ar is recirculated by two turbo-pumps. The working pressure is about 1.3 mTorr for $^{238}\text{U}^{5+}$ at 2.875 MV.
The ions with positive charge states are accelerated a second time by the same potential. The High Energy (HE) magnet, efficiently removes molecular break-up products (De Cesare et al., 2010a,b). The double focusing 90° HE bending magnet has \( r = 1.27 \text{ m} \), \( ME/q^2 = 176 \text{ MeV·amu}/e^2 \) and \( M/\Delta M = 725 \), with slit opening of \( \pm 1 \text{ mm} \) both at object and image points. The two 45° electrostatic spherical analyzers (\( r = 2.54 \text{ m} \) and gap = 3 cm) are operated up to \( \pm 60 \text{ kV} \); energy resolution is \( E/\Delta E = 700 \) for typical beam size. A switching magnet (\( B_{\text{max}} = 1.3 \text{ T} \), \( r = 1.760 \text{ m} \) and \( ME/q^2 = 252.5 \text{ MeV·amu}/e^2 \) at the 20° exit) is positioned after the ESA. Finally the selected ions are counted in an appropriate particle detector, either a surface barrier detector or a telescope ionization chamber. The control of the entire system, is handled by the AccelNet computer based system via CAMAC interfaces or Ethernet, and the acquisition system is either AccelNet itself or FAIR (Fast Intercrate Readout) system, (Ordine et al., 1998).

### 3.1.1 CIRCE actinides measurement procedures

In this paragraph a description of the various steps of the \( ^{238}\text{U} \) and \( ^{239}\text{Pu} \) isotopes measurement are given. The relative abundance of \( ^{238}\text{U} \) in environmental samples is several order of magnitude (up to 13) larger than the \( ^{236}\text{U} \). For this reason, while the number of events of \( ^{236}\text{U} \) are measured in the final detector, \( ^{238}\text{U} \) is measured as current in the high energy side. For the \( ^{239}\text{Pu} \) isotopes, since no natural and so abundant isotopes exist, they are all measure in the final detector. Before performing measurements of samples, a tuning of the transport elements up to the final detector is made by setting the accelerator parameters to the detection of \( ^{238}\text{U} \). Then the MBS, the TV and the high energy ESA are scaled to select the rare isotopes. The sample preparation provides material that is sputtered as \( ^{238}\text{U}^{16}\text{O}^- \) and \( ^{239}\text{Pu}^{16}\text{O}^- \). The negative molecular ions, ex. \( ^{238}\text{U}^{16}\text{O}^+ \), are accelerated to an injection energy of \( E_{\text{inj}} = 50 \text{ keV} \). To select different masses without changing the magnetic field, the energy of the ions inside the injection magnet is varied by applying an additional accelerating voltage to the bouncing system. The injected \( ^{238}\text{U}^{16}\text{O}^- \) ions are accelerated by the positive high voltage towards the stripper, where they loose electrons and gain high positive charge states. The positive ions are, then, accelerated a second time by the same potential in the high energy tube of the tandem. This for \( ^{238}\text{U}^{5+} \) results in an energy of \( E = 17.3 \text{ MeV} \) with a terminal voltage of \( V = 2.900 \text{ MV} \). Ar is recirculated in the terminal stripper by two turbo-pumps; the working pressure is about 1.3 mTorr for \( ^{238}\text{U}^{5+} \) at 2.875 MV (De Cesare et al., 2010b) and the stripping yield achieved for \( ^{238}\text{U}^{5+} \) achieved is around 3.1%.

Molecular break-up products with mass over charge ratio \( M/q \) different from that of the wanted ion, are removed by the combination of the high energy (HE) magnet and an electrostatic analyzer (ESA) whose object point is the image point of the analyzing magnet. For heavy ion measurements, the object and image slits of the injection magnet are closed to \( \pm 1 \text{ mm} \), the slits of the analyzing magnet are closed to \( \pm 2 \text{ mm} \) and a collimator of 4 mm diameter is positioned in the beam waist at the 20° beam line.

The tuning procedure at CIRCE is made by the optimization of HE magnet and ESA in the high-energy side: they are optimized by maximizing the \( ^{238}\text{U}^{5+} \) current in the Last Faraday Cup (LFC). The transmission efficiency between the HE magnet and LFC at 20° is 80 %, with the 4 mm collimator in.

Once the setup for the pilot beam \( ^{238}\text{U}^{5+} \) is found, the voltage at the chamber of the injection magnet, the terminal voltage and the voltage of the ESA are scaled to transmit \( ^{236}\text{U}^{5+} \). In order to measure the \( ^{236}\text{U}^{238}\text{U} \) ratio, the measurement procedure is composed of three automatic steps:
1. measurement of $^{238}\text{U}^{5+}$ current at the high energy side in FC04.
2. the voltage on the magnet vacuum chamber, the terminal voltage and the ESA are then scaled to transmit $^{236}\text{UO}^{2-}$ and a measurement of the count rate of $^{236}\text{U}^{5+}$ in the detector is performed.

3. repetition of step 1

Steps 1 and 3 are necessary to estimate, by linear interpolation, the value of $^{238}\text{U}^{5+}$ current at high energy side which would be measured simultaneously with $^{236}\text{U}^{5+}$ counting. In order to measure the $^3\text{Pu}$ isotope ratios, the measurement procedure is composed of automatic steps:

1. tune the beam with the $^{238}\text{U}^{5+}$ current up to LFC.
2. the voltage on the magnet vacuum chamber, the terminal voltage and the ESA are then scaled to transmit $^3\text{PuO}^{2-}$ and a measurement of the count rate of $^3\text{Pu}^{5+}$ in the detector is performed.
3. repetition of step 2 for all the plutonium isotopes are needed (ex. $^{242}\text{Pu}^{5+}$ spike for 18 s, $^{240}\text{Pu}^{5+}$ for 60 s and $^{239}\text{Pu}^{5+}$ for 30 s).
4. repetition of step 3 for 3 times.

### 3.1.2 CIRCE actinide results

Before the installation of a dedicated actinides beam line at CIRCE, preliminary results for the $^{236}\text{U}/^{238}\text{U}$ background ratio level at 0° line, routinely used for $^{14}\text{C}$ measurements, was of the order of $10^{-9}$ (De Cesare et al., 2010a). The measurement was obtained with the "K. k. Uranfabric Joachimsthal" sample, the VERA in-house U standard, $(6.98\pm0.32)\times10^{-11}$ (Steier et al., 2008).

The main upgrade so far has been the addition of a switching magnet placed 50 cm after the exit of the high-energy ESA. The position of the magnet was decided by means of COSY infinity (Makino & Berz, 1999) magnetic optics simulation (De Cesare et al., 2010a), Fig. 2. This magnet provides a supplementary dispersive analyzing tool.

The abundance sensitivity results, using a 16-strip silicon detector, have shown that, in the upgraded CIRCE heavy ions beamline after the switching magnet installation, a background level $<5.6\times10^{-11}$ has been reached, Fig. 3, compared to $3.0\times10^{-9}$ obtained previously (De Cesare et al., 2010b; Guan, 2010). Although most of the $^{238}\text{U}$ are suppressed at the injector side, by the analyzing magnet and electrostatic analyzer, a small fraction of this intense beam can still interfere with the $^{236}\text{U}$ measurement. The main reasons for this "leakage" of interfering ions are charge exchange processes due to residual gas in the system. Scattering on the residual gas, electrodes, slits or vacuum chamber walls can also allow the background to pass a filter. However, the scattering cross-section is in the order of $10^{-20}$ cm$^2$ whereas the cross section of charge changing is $10^{-16}$-10$^{-15}$ cm$^2$ (Betz, 1972; Vockenhuber et al., 2002).

Moreover, in the upgraded CIRCE heavy ions beamline, after the TOF-E installation, a background level of about $2.9\times10^{-11}$, summing over the central six strips, has been reached, compared to $5.6\times10^{-11}$ obtained with a 16 strip silicon detector alone. This small background reduction is attributed to the 1.6 ns time resolution mainly due to the thickness of the 4 $\mu$g/cm$^2$ LPA (Maier-Komor et al., 1997; 1999) carbon foil, (De Cesare, 2009).

The CIRCE laboratory is not so far from the two systems (ANU and VERA) that provide the best $^{236}\text{U}/^{238}\text{U}$ isotopic ratio sensitivity of $10^{-13}$, in samples including about 1 mg of U.
Fig. 2. The COSY infinity magnetic optics simulation is shown, where the development of two beams has been analyzed, starting from the waist of the high-energy magnet with a relative energy difference of $\Delta E/E = 0.001$ (corresponds to the resolution of the ESA). The adopted beam profiles are approximately Gaussian, with a halfwidth of 0.15 cm. A maximum divergence of 3 mrad was assumed. Simulations were performed for different geometric configurations. The distance ESA-SM (energy electrostatic analyzer-switching magnet), SM-DB (switching magnet-magnetic quadrupole doublet) and DB-FP (Focal Plane = the doublet focusing position) are shown in the upper part. The density relative frequency in function of beam distance in the x-plane is shown in the lower part. The central (solid line) peak is $^{236}\text{U}^{5+}$ and the dashed and dotted are the $^{238}\text{U}$ beams in the two opposite x positions, where the dashed one is not shown in the simulation.

An overview of the planned upgrade of the CIRCE system using a TOF-E system, with a flight path of 3 m and a thinner DLC carbon foil, 0.6 $\mu$g/cm$^2$ is described in (De Cesare, 2009). Regarding the concentration sensitivity results, a 4$\mu$g uranium concentration sensitivity has been reached using only with the 16 strip silicon detector. That correspond to about 40 fg of $^{236}\text{U}$ and $10^9$ $^{236}\text{U}$ atoms for a sample with isotopic ratio of $10^{-9}$ (De Cesare et al., 2011).

For the $^{239}\text{Pu}$ concentration sensitivity results, the uranium background corresponding to the $^{239}\text{Pu}$ settings is at the level of 1 ppb. This is to be compared with the 10 ppm of ANSTO and 100 ppb of ANU. The CIRCE Lab. has at present a $^{239}\text{Pu}$ sensitivity level less than 0.1 fg, since 500 ng of uranium is required to produce an apparent $^{239}\text{Pu}$ concentration of 0.1 fg (De Cesare et al., 2011); for the Pu background level, CIRCE is one of the best system in the world.
Fig. 3. Normalized counts (counts in the detector in 300 s over FC04 current corrected for the transmission $\sim 80\%$ between FC04 and LFC) versus horizontal position of the 16-strip silicon detector. Ch = 3.625 mm is the distance between the center of two adjacent strips. A photo of the 16-strip detector is also shown. The bigger peak represents the position on the detector of the $^{236}$U obtained with a spike sample; the nominal ratio is $^{236}$U/$^{238}$U $\sim 1 \times 10^{-8}$. The lower $^{236}$U peak is obtained with the KkU VERA in house U standard, see text. The arrow indicates that the normalized counts at that position are lower than $1 \times 10^{-2}$ counts/nA.

3.2 ANU system

The ANU AMS system is based on a 15MV-tandem accelerator (Fifield et al., 1996). The high terminal voltage is required to apply certain techniques of isobar separation effectively, this makes the ANU tandem the best suited accelerator for the heavier isotopes e.g., $^{36}$Cl and $^{53}$Mn (Winkler, 2008). When the lower energy is necessary, for $^{236}$U and $^{243}$Pu isotopes, sections of the accelerator tube are shorted out, in order to optimize the ion optics for maximum transmission.

The pre-treated sample material (a few mg is pressed into a 1 mm diameter Al cathode and put in the ion source) itself is analyzed by two mass spectrometers which are coupled to the tandem accelerator. A schematic layout of the ANU 15 MV tandem facility is shown in Fig. 4. The caesium sputter ion source is a 32-sample MSNICS. This multi-cathode arrangement allows for measuring many samples without opening the source or employing a more complicated single cathode exchange mechanism. A total injection energy of 100 keV was used and $\sim 20 \, nA$ of $^{238}$U$^{16}$O$^-$ molecular ions are mass rigidity selected by the 90° double focusing Low Energy (LE) injection magnet ($r = 0.83 \, m$, $B_{max} = 1.3 \, T$, $ME/q^2 \sim 56 \, MeV$-amu/\(e^2\)). This allows high resolution mass analysis for all stable isotopes in the periodic table. In contrast to the CIRCE system, there is no electrostatic analyzer, and hence the
ANU 14 UD Accelerator
- 15 MV Tandem Accelerator

Fig. 4. Schematic lay out of the 15 MV ANU 14-UD (Units Doubled) Accelerator and the $^{236}\text{U}$ and $^{243}\text{Pu}$ isotopes detection line. The Switching Magnet, the Wien Filter, the start and the stop TOF-E detector, the Ionization Chamber, the magnetic quadrupole doublet and the Gas Magnet are shown in the line. C denotes the position-Faraday Cup, A denotes the Aperture of $1.5 \times 4.0 \text{ mm}^2$ and the arrows indicate a slits system. The Accelerator is vertical up to the switching magnet that is indicate with a cross.
low-energy sputter tail is not removed prior to injection into the accelerator. For this reason, it is preferred to tune the system with 232Th16O− rather than 238U16O− (see next section). A beam profile monitor (BPM) before the magnet and Faraday cups after the magnet (LE-Cup and Tank-Cup) are used to monitor the beam during the tuning. The injection beam line also features an electrostatic chopper that allows to reduce the beam intensity in cases where the beam currents are too high for injection into the tandem accelerator or counting rates that would be too high for the detector (e.g. 234U). An electrostatic quadrupole and steerers are available to have the ions pass on an optimum trajectory for injection into the accelerator.

The terminal is charged by chains made of metal pellets which are isolated from each other by nylon links. The pellets supply a total charging current to the terminal of about 230 μA. The accelerator is contained inside a vessel filled with sulphur hexafluoride (SF6) at a pressure of about 6 bar. The voltage is measured by a generating voltmeter. Regulation is achieved by employing a controlled corona discharge from ground to terminal. Both a gas stripper and a foil stripper are available at the terminal. At the terminal the ions lose electrons in the stripper, where O2 is recirculated by two turbo-pumps; the working pressure is about 1 mTorr for 238U5+ at 3.995 MV. Molecular ions are dissociated and the now atomic ions stripped to higher positive charge states (Litherland, 1980). The choice of charge state for heavy ions depends critically on a compromise between its stripping yield and the capability of the subsequent analyzing magnet to bend such ions. At ~ 4 MV, ME/q^2 is ~ 226 MeV·amu/e^2 for 238U5+ and ME/q^2 is ~ 293 MeV·amu/e^2 for 238U4+; since the double focusing HE magnet reaches a maximal ME/q^2 ~ 225 MeV·amu/e^2, the 5+ represents the lowest charge state which can be bent by the HE magnet. Although the stripping yield to 4+ charge state is higher than 5+, it would be necessary to operate at lower terminal voltage in order to bend the ions. Since the transmission (due to the larger scattering angle) and the energy of the ions at this voltage is lower there is no gain to use the lower charge state.

The ions with positive charge states are accelerated a second time by the same potential. The High Energy (HE) magnet, efficiently removes molecular break-up products. The double focusing 90° HE bending magnet has r = 1.27 m, Bmax = 1.7 T, ME/q^2 ≃ 225 MeV·amu/e^2. A switching magnet (Bmax= 1.5 T, r=2.92 m and ME/q^2≃ 926 MeV·amu/e^2 at the 15° exit) is positioned after the HE magnet. A Wien filter (Bmax= 0.25 T, Vmax= ± 60 kV with a Plate Gap= 3 cm) is employed to remove backgrounds.

Finally the selected ions are counted in a final detector. The control of the acquisition system is handled via Ethernet interfaces.

### 3.2.1 ANU actinides measurement procedures

In this paragraph a description of the various steps of the 238U and 239Pu isotope measurements will be given. The relative abundance of 238U in environmental samples is many orders of magnitude (up to 13 ) larger than the 236U. For this reason, while the number of events of 236U are measured in the final detector, 238U is measured as a current at the high energy side. Before performing measurements of samples, a tuning of the transport elements up to the final detector is required in order to maximize the ion optical transmission. The tuning is made by setting the parameters of the beam line to the detection of 232Th. In order to have a good negative ion yield, molecular negative ions 232Th16O− are extracted from the ion source. The negative molecular ions, 232Th16O−, are accelerated to injection energy of Einj= 100 keV. The injected ions are accelerated by the positive high voltage towards the gas stripper, where they lose electrons and gain high positive charge states. The positive ions are then accelerated a second time by the same potential in the high energy tube of the tandem. For 232Th5+,
this results in an energy of E= 24.424 MeV with a terminal voltage of V= 4.098 MV. The stripping yield is the ratio between the $^{232}$Th$^{5+}$ beam current at the Faraday cup after the analyzing magnet (St-C) divided by 5 and the $^{232}$Th$^{16}$O$^-$ current measured at the entrance to the accelerator (T-C), and is about 3%. Molecular break-up products with mass over charge ratio M/q different from that of the wanted ion are removed by the analyzing magnet and switching magnet. The Wien filter is employed to remove backgrounds which have the same ME/q² as the ions of interest but different velocities in the actinides line. For heavy ion tuning, the object and image slits of the injection magnet are closed to ± 1.25 mm and an aperture of 1.5 × 4.0 mm² is used if high selectivity is required just after the Wien filter. For actual measurements, the object and image slits of the injection magnet are opened to ±2 mm, the slits of the analyzing magnet are opened to ±3 mm and the aperture is out.

For Uranium measurements, once the setup for the pilot beam $^{232}$Th$^{5+}$ is found, the fields of the injection magnet, the terminal voltage of the accelerator and the electric field of the Wien filter are scaled to $^{238}$U$^{5+}$ for a fine tuning and then to the other wanted masses. For $^{236}$U/$^{238}$U, the measurement procedure is composed of two loops of three steps. Each loop consists of integration of the $^{238}$U$^{5+}$ beam current for 10 s in the L-C, counting of $^{236}$U$^{5+}$ ions for 5 min in the TOF-E system and a final $^{238}$U$^{5+}$ integration. For $^{233}$U (tracer), $^{234}$U and $^{236}$U, the measurement procedure is composed of two loops of four steps. The isotope sequence would usually start with the reference isotope $^{233}$U followed by $^{234}$U and $^{236}$U, and finishing with $^{235}$U. All of them are counted with the TOF-E system. The typical counting intervals were 1 minute for $^{233}$U, 1 minute for $^{234}$U and 5 minutes for $^{236}$U. For Plutonium measurements, once the setup for the pilot beam $^{233}$Th$^{5+}$ is found, since $^{236}$U$^{5+}$ may cause interference for $^{239}$Pu$^{5+}$, the fields of the injection magnet, the terminal voltage of the accelerator and the electric field of the Wien filter are scaled to the Pu wanted masses, $^{236}$Pu, $^{240}$Pu and $^{245}$Pu (tracer). The measurement procedure is composed of two loops of four steps; the isotope sequence would usually start with the reference isotope $^{242}$Pu followed by $^{246}$Pu and $^{239}$Pu, and finishing with $^{245}$Pu. All of them are counted with a multiple electrode ionization chamber that is routinely used for measurements of Pu isotopes. The typical counting intervals were 1 minute for $^{242}$Pu, 5 minutes for $^{240}$Pu and 3 minutes for $^{239}$Pu.

3.2.2 Detection systems and ANU actinide results

Although most of the $^{238}$U are suppressed at the injector side and by the analyzing magnet and Wien filter, a small fraction of this intense beam can interfere with the $^{236}$U measurement even if the expected separation in the ion-optical filters is large, paragraph 3.1.2. For this reason the detection of the $^{238}$U at ANU is made with a TOF-E detection. The configuration of the TOF detection system is as follows (Wilcken, 2006; Winkler, 2008); the start detector assembly is based on a MCP and a foil is placed at an angle of 45° to the beam. The MCP detects the backscattered electrons from a 0.6 µg/cm² thick diamond-like carbon (DLC) (Lichtenstein et al., 1999; 2002; 2004; 2006) foil that is used in the start detector to minimize scattering. The stop detector is a 200 mm² silicon surface barrier detector which also provides a total energy signal. The foils are mounted on a Cu mesh with a transparency of ~ 75%. The MCP detector was operated with the anode at ground, the accelerating grid and the front face of the MCP at -1.8 kV, and the carbon foil at -2.8 kV. The presence of the foil, which is oriented at 45° to the beam, has two important consequences for the system. First, it causes scattering, which if through a large-enough angle can cause ions to miss the stop detector. This can be minimized by using the thinnest possible foil. Secondly, the 45° tilt introduces differences in path length.
and therefore also in flight time due to the finite size of the beam at the start detector. The effect of the flight path variations on the resolution of the system is minimized by using an aperture that is 3.5 mm wide in the horizontal plane. This is attached on top of the grid-foil assembly.

For plutonium measurements no interfering ions exist; an ionization chamber is suitable for such a detection. The ANU configuration of the ionization chamber (Fifield et al., 1996; Wilcken, 2006; Wilcken et al., 2008) are the following: ~ 50 torr of propane is used as the detector gas and the window is a 0.7 μm thick Mylar foil. Applied voltages are: cathode ≃ -600 V, detector window ≃ -300 V, first grid at ground, second grid at ≃ +200 V and anode ≃ +600 V. The energy of the 239Pu5+ ions is ~ 24.5 MeV. At this energy, the range of the plutonium ions in the ANU detector is ~ 35 mm, which is roughly 18% of the length of the detector. The energy loss and straggling in the detector window are approximately 4.5 MeV and 450 keV, respectively. In addition, according to the manufacturer, a typical value for the surface roughness of the Mylar window is 38 nm, which is 5% of the thickness of the window and contributes an additional 140 keV of straggling. All of these result in an energy resolution of ~ 4%.

Regarding the abundance sensitivity results, the ANU is the best system in the world together with VERA laboratory (Steier et al., 2010). The ANU is able to obtain values of 236U/238U ≃ 10^-13 (Wilcken et al., 2008), in samples including about 1 mg of U. This results is obtained with a time of flight of 2.3 m.

Preliminary results have been obtained with a 6 m flight path; the longer flight path confers a substantial improvement in the ability to separate 235U and 236U with little reduction in efficiency (Fifield, 2011).

The concentration sensitivity limit is of the order of ~ 1 μg of uranium.

As regard the 239Pu concentration sensitivity results, the uranium background at the 239Pu settings is at the level of 100 ppb of the uranium concentration, i.e. 1 ng of uranium in the sample results in a background equivalent to 0.1 fg of 239Pu (Fifield, 2008).

4. Summary and conclusion

The actinides detection technique described in this chapter can be applied in the assessment of contaminations from nuclear facility and used as sensitive fingerprints of programmed and accidental releases; a more general goal of this technique is to provide an ultrasensitive diagnostic tool for a variety of applications to the international community. Moreover the origin of actinides are discussed as well as the potential of actinides to serve as a tracer for geomorphologic processes.

The sensitivity of the different actinides measurements method and the peculiarity of the AMS technique with respect to AS and CMS techniques have been illustrated. Furthermore the principles and methodology of heavy-element AMS as applied to U and Pu isotopes, and the ways in which these have been implemented in various laboratories around the world, have been discussed. In particular the measurement procedures and the concentration and abundance sensitivity results of two systems, CIRCE and ANU, have been described in more details.

Those are two of the few systems in the world able to perform such measurements; the CIRCE is the only one in Italy.

The CIRCE system is at level of ~10^-12 236U/238U isotopic ratio sensitivity which is still one order of magnitude higher then the ANU and VERA systems.
As future plan the CIRCE actinides group foresees to reach and exceed this sensitivity ratio goal with the new upgrade: the utilization of a TOF-E system with a thinner carbon foil and, if necessary, with a longer time of flight.

Regarding the Plutonium background results, the CIRCE is one of the best systems in the world; it is at the level of 1 ppb. This is to be compared with ANSTO where the uranium background is at the level of 10 ppm, and the ANU system where it is at the level of 100 ppb. The CIRCE laboratory has at present a $^{239}$Pu sensitivity level less than 0.1 fg.

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Advances in reactor designs, materials and human-machine interfaces guarantee safety and reliability of emerging reactor technologies, eliminating possibilities for high-consequence human errors as those which have occurred in the past. New instrumentation and control technologies based in digital systems, novel sensors and measurement approaches facilitate safety, reliability and economic competitiveness of nuclear power options. Autonomous operation scenarios are becoming increasingly popular to consider for small modular systems. This book belongs to a series of books on nuclear power published by InTech. It consists of four major sections and contains twenty-one chapters on topics from key subject areas pertinent to instrumentation and control, operation reliability, system aging and human-machine interfaces. The book targets a broad potential readership group - students, researchers and specialists in the field - who are interested in learning about nuclear power.

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